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Neutron imaging and tomography with mcps

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ABSTRACT: A neutron imaging detector based on neutron-sensitive microchannel plates (mcps) was constructed and tested at beamlines of thermal and cold neutrons. The mcps are made of a glass mixture containing $^{10}\text{B}$ and natural Gd, which makes the bulk of the mcp an efficient neutron converter. Contrary to the neutron-sensitive scintillator screens normally used in neutron imaging, spatial resolution is not traded off with detection efficiency. While the best neutron imaging scintillators have a detection efficiency around a percent, a detection efficiency of around 50% for thermal neutrons and 70% for cold neutrons has been demonstrated with these mcps earlier.

Our tests show a performance similar to conventional neutron imaging detectors, apart from the orders of magnitude better sensitivity. We demonstrate a spatial resolution better than 150 µm. The sensitivity of this detector allows fast tomography and neutron video recording, and will make smaller reactor sites and even portable sources suitable for neutron imaging.

KEYWORDS: Electron multipliers (vacuum); Neutron detectors (cold, thermal, fast neutrons); Neutron radiography; Vacuum-based detectors

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1 Introduction

Detectors for thermal neutron imaging need to combine a good spatial resolution with a field of view sufficiently wide for the sample under study. The most conventional technology combining these requirements is a thin neutron-sensitive scintillator, read by a highly sensitive scientific CMOS or CCD camera. The scintillating material is usually ZnS:Cu, emitting around 531 nm, near the optimum of CMOS sensitivity. This scintillator is used in powder form, also known as the green phosphor P31, mixed with $^6$LiF powder. The $^6$Li (highly enriched) here acts as the neutron converter through the fission reaction $n(^6Li, α)^3H$. Both fission fragments have a range of few tens of microns in the compound. The resolution is limited by the thickness of the $^6$LiF/ZnS:Cu layer; with both constituents in powder form the layer has a white appearance, and light scattering between grains limits the resolution to approximately the thickness of the layer.

This way of constructing an imaging detector has many benefits. It offers great flexibility: $^6$LiF/ZnS:Cu is inexpensive, and can be applied by brushing or spraying on aluminum substrates of any size, which are transparent to neutrons. If the thickness of the layer is well controlled, a resolution of better than 50 microns can be attained. All elements in the mixture have a low atomic number, which makes it rather insensitive to the gamma background found in every neutron beam environment.

A difficulty with this technique, however, is the limited sensitivity, and the fact that sensitivity is necessarily traded off with spatial resolution. Scintillators with thickness from fifty to a few hundred microns typically stop only a few percent of thermal neutrons. Moreover, ZnS:Cu has a long afterglow,\(^1\) complicating tomography or other situations where multiple consecutive images are taken. An alternative neutron scintillator sometimes used, Gadox (Gd$_2$O$_2$S), does not have this afterglow, but due to the high atomic number of gadolinium ($Z = 64$) it is also quite efficient as a gamma scintillator, contributing to the background.

\(^1\)ZnS:Cu is also known as the glow-in-the-dark phosphorescent in many toys and cosmetics.
In this work we present a neutron imager based on a neutron-sensitive microchannel plate (MCP) coupled to a fast phosphor. The physical mechanism by which neutron capture in the MCP leads to a scintillation image on the screen is quite different from the case with neutron-sensitive scintillators. But since in both cases the result is an image on a scintillating screen, our MCP imager can be placed in the position of a neutron scintillator, and be read with the same mirror/lens/camera setup already in use. This is indeed how we have tested our prototype. The benefit of this system compared to neutron scintillators is primarily a high spatial resolution that is combined with a good detection efficiency; efficiency and resolution are not traded off.

The possibility to make MCPs of a modified glass composition to make them neutron sensitive was proposed and demonstrated by Fraser in 1990 [1]. This concept was further studied and optimized by Tremsin et al., who eventually demonstrated a detection efficiency of 50% for thermal neutrons and 70% for cold neutrons [2], and a spatial resolution better than 15 µm [3]. These same optimized MCPs are the basis of the neutron imager presented here.

2 An MCP-based neutron imager

The neutron sensitive MCPs we use are made of a glass mixture modified to include enriched $^{10}$B and natural Gd, both with a high neutron capture cross-section. Neutron capture triggers a nuclear reaction: either $n(^{10}B, α) _{7}Li + γ$ (fission) or $n(^{157}Gd, ^{158}Gd)e^- + γ$ (internal conversion). The fission fragments or conversion electrons have a range of a few microns in this glass. With an 8 µm pore diameter and a 10 µm pore spacing, this range is sufficient to reach the nearest pore, where electrons are emitted in the vacuum. This starts a cascade of secondary emissions that is the normal working principle of MCPs. While most detectors made with these MCPs are read out electronically, with high granularity strip or pixel anodes, the detector presented here is coupled to a phosphor screen and read out optically by a camera. Figure 1 shows a drawing of the assembly.

The imager used for these tests used a neutron sensitive MCP followed by a regular MCP; this configuration allows a high gain, but widens the point spread function, thus affecting the imaging resolution. A fiber optic faceplate coated with an indium tin oxide (ITO, a transparent conductor) electrode serves as the substrate on which the phosphor screen is deposited. The phosphor used is

![Figure 1](image-url)
Figure 2. The neutron imager in the CG-1D beam line of HFIR at Oak Ridge National Lab. On the bottom left the controller unit of the turbo pump and the high voltage power supply are visible.

P46 ($Y_3Al_5O_{12}$:Ce), a fast phosphor emitting in the green. The MCP/phosphor screen assembly is installed in a stainless steel vacuum enclosure, a few millimeters from a sapphire viewport which serves as the neutron entrance window. The remaining volume in the vacuum system is empty, serving only to optimize conductance to the turbo pump, see figure 2. The camera\textsuperscript{2} with its relay lens are mounted on another viewport at the back of the vacuum chamber. This is not ideal, since the camera operates on the neutron beam axis, and is exposed to any neutron flux not absorbed by the MCP; we foresee a mirror mounted at $45^\circ$ to the beam axis in further studies, so the camera and lens will be protected from the neutron flux.

2.1 Acquiring neutron images

The high gain of the double MCP, combined with the good sensitivity and limited well depth of the CMOS sensor ($25000 \text{ e}^-$), causes pixel saturation at long exposure. Therefore several frames of short exposure ($< 1 \text{ s}$) are acquired, which are then averaged to obtain a longer exposure image. The gray value of each pixel of the image must be normalized to a full scale running from the background level (black) to the value obtained with the neutron beam on, but the sample removed (white):

$$I_{\text{norm}} = \frac{I_{\text{raw}} - I_{\text{bkg}}}{I_{\text{ob}} - I_{\text{bkg}}},$$

(2.1)

with the suffixes bkg and ob meaning background and open beam, respectively. This cancels the effects of any non-uniformity of backgrounds, the beam profile and the gain of the MCPs. Figure 3 shows this process and its effect on the image. These operations and all other image processing in this work were done with the open source ImageJ\textsuperscript{4} software.

\textsuperscript{2}A Nocturn gp monochrome CMOS camera from Photonis was used.
3 Results

We will discuss a series of images taken primarily with the aim to reveal some of the performance characteristics of this prototype.

3.1 Gadolinium test mask

The neutron imaging group at the Paul Scherrer Institute in Switzerland has developed high definition gadolinium patterns using a silica wafer as a substrate [5], see figure 4, left. These test masks are helpful tools to quantify specific performance aspects of a detector. The images from this mask discussed below were taken at the CG-1D cold neutron imaging beamline of HFIR at Oak Ridge National Lab.

**Siemens star.** Figure 4 shows a photograph of a wafer with the Gd pattern (left). There are 3 relevant patterns on the 100 mm diameter wafer, the neutron image on the right shows the Siemens star. A few circles mark line pitches as indicated in the figure; with this pattern one can immediately estimate the limiting resolution to be between 200 and 100 µm. Since the pattern has radial lines in...
all directions, it will also easily reveal differences in horizontal and vertical resolution. This could happen due to optical misalignment of the camera and lens behind the phosphor screen, or trivial details such as pump vibrations coupling to the imager. No such difference is seen here.

**Mtf bar pattern.** The pattern of progressively more closely spaced bars (also shown in figure 3) provides a more quantitative way to characterize the detector’s spatial resolution, see figure 5. As the spatial frequency of the bar pattern increases from left to right, the contrast between light and dark shapes diminishes, as clearly shown by the overlaid intensity profile. The relative contrast as function of the spatial density is called the Modulation Transfer Function (MTF), see the curve on the right. The MTF is equal to the Fourier transform of the line spread function, but the bar pattern provides a convenient way of measuring it. The limiting resolution is the finest spacing that can still be resolved; although somewhat arbitrary, one often equates it to the resolution at 5% MTF. Using this standard, the limiting resolution of this detector is just below 150 µm. It is known from other MCP/phosphor screen detectors (e.g. image intensifier tubes) that MCPs have a non-Gaussian line spread function, which causes the MTF to fall rather gradually; scintillator imagers on the other hand have an MTF that drops quite steeply.

**Square grid pattern.** This pattern is a checked square with an even density of lines with varying thickness, see figure 6, left. The line spacing is 1 mm, and the lines are 50, 100 and 150 µm wide. This pattern provides a means of measuring distortions, such as pincushion or barrel distortions. Any such distortions will normally be due to the optics behind the phosphor screen and not caused by the imager itself. A 2D discrete Fourier transform of the image will reveal any distortions clearly, as the pattern of spots will smear out if there is any distortion. To the right of figure 6 is a 2D Fourier transform, showing no distortions. The fact that the active area of 40 mm diameter is rather small compared to many scintillator screens may help here, as well as the absence of a mirror to reflect the image away from the neutron beam axis.

### 3.2 Neutron tomography

Computed tomography is a technique where the benefit of much greater sensitivity compared to scintillator screens is particularly helpful. A sample is mounted on a rotation stage and images are taken at many hundreds of different angles. With the sensitivity of scintillating screens, the...
Figure 6. The square grid pattern (left) can reveal image distortions. A discrete Fourier transform of this image (right) is very sensitive to any such distortions.

Figure 7. Left, an aluminium cylinder with inserts of several other metals. Right, a neutron image of this cylinder.

acquisition of so many images takes many hours, and often needs to be done overnight. With this mcp-based neutron imager we can do a scan of similar quality in two hours or less. Figure 7 shows a test device developed at PSI [6] to evaluate the contrast revealed by a neutron imaging beamline and its detector. It is an aluminum cylinder, with smaller cylindrical inserts of other metals. To the right is a neutron image taken at the imaging beamline of the Reactor Institute of Delft University of Technology; the elements of the inserts are indicated.

Figure 8 shows atomographic reconstruction of the same object. The same algorithms used for X-ray CT are employed to reconstruct a voxel map of attenuation coefficients\(^3\) from the many images, 900 projections in this case. In the center of figure 8 is a histogram of the grey values in the 3D voxel map of the reconstruction, set to a scale of attenuation coefficients. The six different materials present in the sample are clearly visible as six peaks of equal area, except for the aluminum peak from the cylinder itself. The attenuation coefficients of the six peaks are consistent with literature values for these metals. Remarkable is the low attenuation of lead, which has great stopping power for X-rays; for thermal neutrons it is hardly more absorbing than aluminum. This histogram also provides the key to “seeing through” samples, one of the powerful tools tomography offers. If in the 3D reconstruction image one puts limits on what is displayed or not, certain materials can be rendered invisible so as to only show the materials of interest. The right of figure 8 shows the same

\(^3\)The shown reconstruction was done with Octopus 8.8.1.0, a software package designed for X-ray CT.
0.0 0.2 0.4 0.6 0.8 1.0 1.2 1.4 1.6 1.8 2.0
Linear attenuation coefficient (cm\(^{-1}\))
Al Pb Ti Cu Fe Ni
threshold: 0.5 cm\(^{-1}\)

Figure 8. Left, tomographic reconstruction of the test object from figure 7. Right, histogram of greyscales of the 3D voxel map. The horizontal axis expresses the grey values as attenuation coefficient.

reconstruction where only voxels over a threshold of 0.5 cm\(^{-1}\) are displayed. From the histogram it is clear that this should exclude the aluminum cylinder and the lead insert, and show the other four inserts.

4 Conclusions and outlook

Tests done at two different beamlines with different neutron energies (see figure 9) have shown that this first prototype performs well for neutron imaging and tomography. The sensitivity is orders of magnitude higher than conventional scintillator-based imaging detectors. The resolution, although sufficient for most imaging applications, is not yet as good as the best scintillator detectors. There is also room for improvement on practical aspects. After being tested extensively, the stainless steel vacuum system of the imager shown here is strongly activated, contributing significantly to its own gamma background. The setup consists of many parts, pumps, controllers, power supplies etc., all interconnected by a web of cables and hoses.

Figure 9. Neutron images of a Planacon™ MCP-PMT with thermal neutrons (Delft) and with cold neutrons (Oak Ridge). This highlights the difference between thermal and cold neutron imaging: thermal neutrons have deeper penetration, but cold neutrons give more contrast.
An improved MCP based imager, already completed at the time of writing, has a square active area of $100 \times 100 \text{mm}^2$, and attains a limiting resolution of better than $50 \mu \text{m}$. A square-shaped field of view is advantageous when doing tomography, resulting in a cylindrical fiducial volume, rather than the spherical fiducial volume of the round imager in this work. Its choice of materials and construction has eliminated the activation issues, and all loose parts are located in one control unit, with a USB connection to a computer to operate it remotely.

A neutron imaging detector with a sensitivity so much higher than present-day techniques may enable new sorts of imaging studies. Neutron tomography can be done so quickly that many samples can be scanned the same day, making more efficient use of precious beam time. Neutron imaging can also be applied to dynamic processes, opening the door to neutron video recording.

The fact that many times less neutron flux is integrated to attain a certain image quality also comes with benefits. Samples activate less, proportionally to exposure time. Rare artifacts and valuable museum pieces can be imaged and still return to their owner. Small, low power nuclear reactors running on conventional low-enriched uranium become suitable neutron sources for imaging. The images in this study taken at the reactor in Delft are a case in point. We are exploring the possibility of neutron imaging with neutron generators, which may take neutron imaging from large scale user facilities to labs in academia and industry.

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